



Regulatory Guide

Content of Nuclear Facility Construction and Operating Licence Applications (FANR-RG-001)

Version 1

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Table of Contents

Definitions	3
Article (1)	3
Requirement	5
Article (2)	5
Purpose.....	6
Article (3)	6
Structure.....	6
Article (4)	6
Content	6
Article (5)	6
Certification and Information Control	8
Article (6)	8
Specific Guidance and Content.....	8
Article (7)	8
Safety Analysis Report Content.....	9
Article (8)	9
Format of the Safety Analysis Report	18
Article (9)	18
References.....	19
Article (10)	19



Basic Principle of Regulatory Guides

Regulatory Guides are issued to describe methods and/or criteria acceptable to the Authority for meeting and implementing specific requirements in FANR regulations. Regulatory guides are not substitutes for regulations, and compliance with them is not required. Methods of complying with the requirements in regulations different from the guidance set forth by the regulatory guide can be acceptable if the alternatives provide assurance that the requirements are met.

Definitions

Article (1)

For purposes of this regulatory guide, the following terms shall have the meanings set forth below. Other capitalised terms used but not defined herein shall have the meaning ascribed to them in Article 1 of the Federal Law by Decree No. 6 of 2009 Concerning the Peaceful Uses of Nuclear Energy (the Law):

Acceptance Criteria	Criteria that a system, structure or component must meet in order to be accepted by a user or by the established authority.
Additional Protocol	The 2009 Protocol Additional to the Agreement between the UAE and the IAEA for the application of Safeguards in connection with the Treaty on the Non-Proliferation of Nuclear Weapons (2003).
Independent Safety Verification (ISV)	A written verification performed by adequately qualified and experienced individuals, who did not participate in the original Safety Assessment, to determine whether the approach taken in conducting such Safety Assessment was reasonable and in accordance with international best practice.
Inspection, Tests, and Analysis	Activities that are conducted under specified conditions and/or assumptions to verify that a given system, structure or component meets its acceptance criteria.



Initial Test Programme (ITP)

A programme consisting of pre-operational and initial start-up tests. Pre-operational testing consists of those tests conducted following completion of Construction and Construction-related Inspections and tests, but prior to fuel-loading, to demonstrate to the extent practical the capability of SSCs to meet the performance requirements to satisfy the design criteria.

Initial start-up testing consists of those test activities that are scheduled to be performed during and following fuel-loading. These activities include fuel loading, pre-critical tests, initial criticality, low-power tests, and power-ascension tests that confirm the design bases and demonstrate to the extent practical that the nuclear power plant will operate in accordance with the Design and is capable of responding as designed to anticipated transients and postulated Accidents as specified in the Safety Analysis Report (SAR).

Limited Construction Licence

The Limited Licence for Parts and Stages of Construction of a Nuclear Facility issued by the Authority granting authorisation to carry out different parts and stages of Construction of a Nuclear Facility prior to the issuance of a Construction Licence.

Operational Limits and Conditions

A set of rules setting forth parameter limits, the functional capability and the performance levels of equipment and personnel approved by the regulatory body for safe Operation of an authorised Facility.

Postulated Initiating Event (PIE)

An event identified in Design as leading to anticipated Operational occurrences or Accident conditions. A PIE is not an Accident itself, but the event that initiates a sequence and leads to an Operational occurrence, a Design basis Accident or a severe Accident depending on the additional failures that occur including equipment failures such as pipe breaks, human errors, human-induced events and natural events.



Probabilistic Risk Assessment (PRA)

A comprehensive, structured approach to identifying failure scenarios constituting a conceptual and mathematical tool for deriving numerical estimates of risk.

Level 1 comprises the Assessment of failures leading to the determination of the frequency of core damage.

Level 2 constitutes the Assessment of containment response and leads to the determination of frequency of containment failure resulting in release to the environment of a given percentage of the reactor core's inventory of radionuclides.

Safety Analysis Report (SAR)

The detailed demonstration of the Safety, security and safeguards of a Nuclear Facility presented in the form of an integrated report that presents the necessary and sufficient information in support of the Licence application for authorisation of the Regulated Activity requested.

Safety Evaluation Report (SER)

The regulatory review and assessment of the Construction Licence application and the operating Licence application, which is presented in the form of an integrated report that summarises the review and assessment performed by or for the regulatory body and provides a clear conclusion about the Safety of the authorised activity.

Structures, Systems and Components (SSCs)

A general term encompassing all the elements of a Facility or Activity which contributes to protection and Safety, except human factors. Structures are the passive elements such as buildings, vessels and shielding. A System comprises several components assembled in such a way as to perform a specific active function and a Component is a discrete element of a system.

Requirement

Article (2)

The Law requires any applicant/ Licensee who proposes to construct or operate a Nuclear Facility to request and receive the appropriate Licence from the Authority. Prior to the Licence being granted, the applicant/ Licensee is required to submit, in accordance with the relevant regulations (References 1 and 2), an application for the requested Licence including detailed evidence of Safety and Nuclear Security and information relevant to safeguards. The



application will be reviewed and assessed by the Authority. The information contained in the Licence application forms the primary basis for the review and Assessment by the Authority, which determines the decision whether to grant or refuse the requested Licence.

Purpose

Article (3)

This regulatory guide sets forth the Authority's guidance for the content of the detailed information to be submitted by the applicant/ Licensee in support of an application for a Licence, as required by the Law and the applicable regulations (References 1 and 2), to construct or to commission/ operate a Nuclear Facility.

Structure

Article (4)

This regulatory guide is partly based on the structure and content outlined by the IAEA Safety Guide GS-G-4.1 (Reference 3), US Nuclear Regulatory Commission (NRC) Regulatory Guide 1.206 (Reference 4) and USNRC Regulatory Guide 1.70 (Reference 5). The guidance herein applies to documentation submitted for the Construction Licence application and for the operating Licence applications. The guidance is also applicable to applications for Limited Construction Licences.

Contents

Article (5)

1. The applicant/ Licensee should provide in its application information demonstrating that the requested Regulated Activity will be carried out in accordance with relevant State Laws, international nuclear treaties, and the Authority's regulations.
2. In the licensing of a new plant the preliminary and final SARs are important documents, compiled by the applicant/ Licensee, which the regulatory authority uses in assessing the adequacy of the plant Design. The SAR, whether preliminary for a Construction Licence or final for an operating Licence, should provide the detailed evidence of safety, security, and safeguards to be reviewed and assessed by the Authority and should provide the primary basis for the Authority's decision to grant a Licence with appropriate conditions, or to refuse such a Licence.



3. The applicant/ Licensee should provide sufficiently detailed, accurate and complete information in the Licence application to enable the Authority to carry out a review and Assessment upon which is based the decision to grant or refuse the requested Licence.
4. The applicant/ Licensee should schedule coordination meetings with the Authority before completing or submitting the Licence application to confirm its understanding of the Licence conditions, to identify expectations, to establish official channels of communications and appraise each other of milestone schedules.
5. The applicant/ Licensee shall submit a Physical Protection Plan as part of the application for a Construction or operating Licence for a Nuclear Facility in compliance with the regulation FANR-REG-08 (Reference 10).

The Physical Protection plan for the Construction phase shall implement especially the Articles (5) and (6) of FANR-REG-08 (Reference 10). The applicant/ Licensee shall submit preliminary information on the Design features of the proposed Nuclear Facility and preliminary information on the Physical Protection of the Operation phase.

The Physical Protection plan for the Operation phase shall describe how the applicant/ Licensee will protect Nuclear Material and the Nuclear Facility against unauthorised removal of Nuclear Material and radiological sabotage up to and including the Design basis threat.

FANR Nuclear Security regulatory guides and the IAEA Nuclear Security Series No. 13 (INFCIRC/225/Rev.5) as well as its implementing guidance provides guidance on the implementation of the Physical Protection for a Nuclear Facility.

6. The applicant/ Licensee must implement the requirements from FANR-REG-09 and FANR-REG-10 (References 11 and 12) that contain more details with regard to safeguards obligations and requirements compared to those from FANR-REG-06 and FANR-REG-14 (References 1 and 2).

According to FANR-REG-10, Article (18) (Reference 12), the Applicant/Licensee shall submit preliminary Design information when applying for a Licence for the Construction of a Nuclear Facility, updated Design information when applying for a Licence for possession, use, manufacture or handling of Nuclear Material and corrected Design information prior to implementing any structural or procedural change to the Facility relevant to Nuclear Material Accountancy and Control.

The FANR Regulatory Guide 14 on the Implementation of the System of Accounting for and Control of Nuclear Material at Nuclear Facilities (Reference 13), which, in line with the United



Arab Emirates Safeguards Agreement including subsidiary arrangement and additional protocol to the Safeguards Agreement provides further specific guidance on the implementation of Safeguards obligations and requirements, includes the update of the preliminary design information and incorporates the “Design Information Questionnaire” (DIQ) SG-FM-1098-06 (Reference 14).

Certification and Information Control

Article (6)

1. Information provided in the Licence application should be certified as up-to-date, accurate and complete as attested by a duly authorised Person of the organisation making the application.
2. The applicant/ Licensee may seek to have the Authority classify and restrict access to certain information in the Licence application, and thereby have it withheld from public disclosure. This may include information that is proprietary, or information that should be withheld for reasons relating to security or safeguards. The applicant/licensee should prominently mark such information to be withheld as protected, provide the basis for withholding such information and state the nature and extent of resulting harm, if disclosed.
3. The applicant/ Licensee should provide a list of, and make available upon request of the Authority, the Topical Reports, supplemental technical reports, or other material referenced in the Licence application.

Specific Guidance and Content

Article (7)

1. This regulatory guide makes use of Reference (3) and Reference (4) to provide guidance on the acceptable general content of the SAR. Such use does not imply that the Authority has endorsed any other third-party regulations or standards that may be embedded in these references. Any application for a Licence from the Authority must refer to the relevant State laws, international nuclear treaties, and the Authority’s regulations in conformance with Article (5) above.
2. Where FANR Regulation 06 on an Application for a Licence to Construct a Nuclear Facility requires (FANR-REG-06) “preliminary information”, the applicant/ Licensee should provide sufficient information about the proposed approach to allow a complete Safety review for the Construction activities, and clearly indicate that the information in its existing form is of a preliminary nature.

3. For those sections where the information is fully provided as part of a Licence application, the applicant/ Licensee should provide an update of this information in any subsequent Licence applications for the same nuclear power plant. The content of the application should be consistent with the reference(s) cited. All documents should:
 - be written in English, and
 - use the international system of units or derived units (meters, kilograms, seconds, becquerel, sieverts, etc).
4. The lessons learnt from previous Construction Licence applications and operating Licence applications should be considered in any new Licence application.

SAR Content

Article (8)

1. ***Introduction and General Description of Plant (FANR-REG-06, Article (6), Paragraph 1 and 2 and REG-14, Article (5), Para 1 and 2)***

Part I Section CI.1 of Reference (4) provides guidance on the acceptable general content of this chapter

2. ***Site Characteristics (REG 06, Article (6), Para 5 and REG 14, Article (5), Para 5)***

Part I Section CI.2 of Reference (4) provides guidance on the acceptable general content of this chapter

3. ***Design of Systems, Structures, Components, and Equipment (REG 06, Article (6), Para 6 and 7 and REG 14, Article (5), Para 6 and 7)***

Part I Section CI.3 of Reference (4) provides guidance on the acceptable general content of this chapter

4. ***Reactor (REG 06, Article (6), Para 7 and REG 14, Article (5), Para 7)***

Part I Section CI.4 of Reference (4) provides guidance on the acceptable general content of this chapter

5. ***Reactor Coolant and Connecting Systems (REG 06, Article (6), Para 7 and REG 14, Article (5), Para 7)***

Part I Section CI.5 of Reference (4) provides guidance on the acceptable general content of this chapter

6. **Engineered Safety Features (REG 06, Article (6), Para 6 and 7 and REG 14, Article (5), Para 6 and 7)**

Part I Section CI.6 of Reference (4) provides guidance on the acceptable general content of this chapter
7. **Instrumentation and Controls (REG 06, Article (6), Para 6 and 7 and REG 14, Article (5), Para 6 and 7)**

Part I Section CI.7 of Reference (4) provides guidance on the acceptable general content of this chapter
8. **Electric Power (REG 06, Article (6), Para 6 and 7 and REG 14, Article (5), Para 6 and 7)**

Part I Section CI.8 of Reference (4) provides guidance on the acceptable general content of this chapter
9. **Auxiliary Systems (REG 06, Article (6), Para 6 and 7 and REG 14, Article (5), Para 6 and 7)**

Part I Section CI.9 of Reference (4) provides guidance on the acceptable general content of each chapter
10. **Steam and Power Conversion System (REG 06, Article (6), Para 7 and REG 14, Article (5), Para 7)**

Part I Section CI.10 of Reference (4) provides guidance on the acceptable general content of this chapter
11. **Radioactive Waste Management including Storage prior to Disposal (REG 06, Article (6), Para 14 and REG 14, Article (5), Para 14)**
 - a. Part I Section CI.11 of Reference (4) provides guidance on the acceptable general content of this chapter
 - b. Specific guidance on Storage prior to Disposal (Reference (3) Para 3.215 -3.216)
12. **Radiation Protection (REG 06, Article (6), Para 13 and REG 14, Article (5), Para 13)**

Part I Section CI.12 of Reference (4) provides acceptable guidance for the general content of this chapter



13. Conduct of Operations (REG 06, Article (6), Para 11 and 15 and REG 14, Article (5), Para 11 and 15)

Part I Section CI.13 of Reference (4) provides acceptable guidance for the general content of this chapter

14. Inspection, Test, Analyses and Verification Programmes (REG 06, Article (6), Para 7 and 10; REG 14, Article (5), Para 7 and 10)

- a. Part I Section CI.14 of Reference (4), Chapter 14 of Reference (5) and Reference (6) provide guidance regarding the Initial Test Programme-related information to be included in both the Preliminary SAR and the Final SAR.
- b. FANR-REG-06 describes the expected content of the Preliminary SAR. The Preliminary SAR provides a primary basis for the Authority's review and Assessment whether to grant a Construction Licence and upon which conditions. The Preliminary SAR should contain:
 1. An outline of the initial test programme
 2. A description of the pre-operational test programme and procedures
 3. In Chapter 14 or by reference to other chapters:
 - a) Design data and acceptance criteria for the systems, structures and components (SSC) of the Facility
 - b) Commitments to Inspection, testing and/or analyses to verify that the SSCs will perform adequately in service
 - c) A summary description of the Construction Inspection and test plan that the applicant will implement to provide reasonable assurance that the Facility is constructed in accordance with the Design requirements, to include but not limited to:
 - 1) Programmatic Activities
 - Programmatic activities (e.g. quality assurance, training) employed to ensure adequate Construction
 - The standards or guidance upon which these programmatic activities are based

- How the applicant/ Licensee ensures the organisations and people (on-site and off-site) carrying out Construction activities comply with established programmes
- A description of the positions performing Inspections/ testing and how individuals performing Inspections/ testing are qualified

2) Construction Activities

For items including but not limited to foundations and buildings, structural concrete, piping, pipe supports and restraints, mechanical components, valves, electrical components and systems, electrical and fiber optic cable, Instrumentation and Control components and systems, and welding:

- Which SSCs will receive Inspection and/ or testing, and how these SSCs are selected
- Description of Inspection coverage (e.g.100 percent Inspection, Inspections on a sampling basis, etc.) and if sampling, the justification for sampling as well as an explanation of how samples are chosen
- How Inspection results are documented
- Description of process to protect SSCs (i.e. kept in their as- built condition) once inspected/ tested and accepted

3) Inspection and Test Planning

- Process for scheduling Inspections and tests
- Discussion of hold points based upon confirming adequate Construction

c. FANR-REG-14 describes the expected content of the Final SAR. The Final SAR provides information that was not already detailed in the Preliminary SAR or not submitted to the Authority to grant an Operation Licence by the Authority. The Final SAR should contain:

1. Detailed evidence of Safety that the facility has been constructed and can be operated in accordance with the Design requirements
2. A commitment to provide the basis upon which the applicant/ Licensee has determined that all Construction completed subsequent to submission of the

Final SAR was conducted in accordance with the Design requirements including a summary of results for all Inspections and tests conducted during Construction up to the point of fuel loading.

3. A description of the initial start-up test programme and procedures as also referred to in FANR-REG-16 in Article 24 (Reference 7)

15. *Transient and Accident Analyses (REG 06, Article (6), Para 9 and REG 14, Article (5), Para 9)*

Part I Section Cl.15 of Reference (4) provides guidance on the acceptable general content of this Chapter

16. *Technical Specifications (REG 06, Article (6), Para 12 and REG 14, Article (5), Para 12)*

Part I Section Cl.16 of Reference (4) provides guidance on the acceptable general content of this Chapter

17. *Management of Safety, Security and Safeguards (REG 06, Article (6), Para 4 and REG 14, Article (5), Para 4)*

In this chapter the applicant should describe, consistently with FANR-REG-01 (Reference 8) and FANR-RG-002 (Reference 9), the management arrangements it will establish to ensure the requirements for Safety, security and safeguards will be complied to.

a. Organisation

Paragraph 3.155 of Reference (3) provides guidance on the acceptable general content of this section

b. Management System for Siting, Design, Procurement and Construction

The documentation of the Management System for siting, Design, procurement and Construction activities should include:

1. the policy statements of the organisation;
2. a description of the Management System;
3. a description of the structure of the organisation;
4. a description of the functional responsibilities, accountabilities, levels of authority and interactions of those managing, performing and assessing work;

5. a description of the processes and supporting information that explain how work is to be prepared, reviewed, carried out, recorded, assessed and improved; and;
6. a description of the programme for handling non-conformances and corrective/preventive action.

The documentation should reflect the characteristics of the organisation and its activities, as well as the complexities of processes and their interactions. It does not necessarily need to include, for purposes of this submittal, the underlying procedures associated with the processes except where called out in other paragraphs.

c. Consideration of Safety and Security Culture for Siting, Design, Procurement and Construction

This sub-section should present the organisation's strategy to develop, maintain and enhance a strong Safety and security culture. The information provided should demonstrate that the necessary arrangements are adequate and are in place. These arrangements should be aimed at promoting good awareness of all aspects of Safety and security, and regularly reviewing with staff the level of Safety and security awareness achieved on the site. The organisation should, where possible, determine indicators of Safety culture as well as of security culture and should develop a programme to monitor such indicators. The staff should be consulted on the determination of these indicators and should be kept informed of the outcome of the reviews. Action should be taken in response to any indications of declining Safety and/ or security levels.

d. Consideration of Quality Assurance for Siting, Design, Procurement and Construction

The principal aspects of the quality assurance (QA) system developed for the nuclear power plant should be described in this sub-section. It should be demonstrated that appropriate provisions for QA including a QA programme and audit, review and self-assessment functions have been implemented for all Safety related plant activities. These activities should include siting, Design (including Design changes made during Construction), procurement of goods and services (including the use of contractors' organisations), plant Construction, Inspection and testing.

e. Management System for Commissioning and Operations

The documentation of the Management System for Commissioning activities and Operations should include:



1. the policy statements of the organisation;
2. a description of the Management System;
3. a description of the structure of the organisation;
4. a description of the functional responsibilities, accountabilities, levels of authority and interactions of those managing, performing and assessing work;
5. a description of the processes and supporting information that explain how work is to be prepared, reviewed, carried out, recorded, assessed and improved; and;
6. a description of the programme for handling non-conformances and corrective/preventive action.

The documentation should reflect the characteristics of the organisation and its activities as well as the complexities of processes and their interactions. It does not necessarily need to include (for purposes of this submission) the underlying procedures associated with the processes except where called out in other paragraphs.

f. Consideration of Safety and Security Culture for Commissioning and Operations

This sub-section should present the organisation's strategy to develop, maintain and enhance a strong Safety and security culture during Commissioning and Operations. The information provided should demonstrate that the necessary arrangements are adequate and are in place. These arrangements should be aimed at promoting good awareness of all aspects of Safety and security, and at reviewing regular intervals with staff the level of Safety and security awareness achieved on the site. The organisation should, where possible, determine indicators of Safety culture as well as for security culture and should develop a programme to monitor such indicators. The staff should be consulted on the determination of these indicators and should be kept informed of the outcome of the reviews. Action should be taken in response to any indications of declining Safety and/ or security levels.

g. Consideration of Quality Assurance for Commissioning and Operations

- h. The principal aspects of the quality assurance (QA) system developed for the nuclear power plant should be described in this sub-section. It should be demonstrated that appropriate provisions for QA, including a QA programme and audit, review and self-assessment functions have been implemented for all Safety related plant activities throughout the Commissioning process and Operations.

i. Monitoring and review of Safety performance

Paragraphs 3.15 to 3.16 of Reference (3) provides guidance on the acceptable general content of this section

18. **Human Factors Engineering (REG 06, Article (6), Para 6 and REG 14, Article (5), Para 6)**

Part I Section CI.18 of Reference (4) provides guidance on the acceptable general content of this Chapter

19. **Probabilistic Risk Assessment and Severe Accident Analysis (REG 06, Article (6), Para 9 and REG 14, Article (5), Para 9)**

Part I Section CI.19 of Reference (4) provides guidance on the acceptable general content of this chapter

20. **Physical Protection (REG 06, Article (6), Para 16 and REG 14, Article (5), Para 16)**

The applicant/ Licensee shall include a summary of the Physical Protection plan in the Preliminary SAR and Final SAR.

21. **Safeguards (REG 06, Article (6), Para 17 and REG 14, Article (5), Para 17)**

The applicant/ Licensee shall include a summary of the Safeguards implementation that should include information about the Facility Safeguards Plan in the Preliminary SAR and Final SAR.

22. **Decommissioning and End-of-Life Aspects (REG 06, Article (6), Para 18 and REG14, Article (5), Para 18)**

The applicant/ Licensee should develop this section appropriately, based on the licensing stage (Preliminary SAR or Final SAR), and appropriate Regulations issued by the Authority, in particular FANR-REG-21 (Reference 15). Paragraphs 3.217 to 3.221 of Reference (3) provide guidance on the acceptable general content of this chapter.

Supplement 1. Reference Nuclear Facility, Modification and Independent Safety Verification (REG 06, Article (6), Para 3 and REG 14, Article (5), Para 3)

Information should be provided about the reference Nuclear Facility including the SARs used by the country of origin (CoO) for its Safety review and the SERs prepared by the regulatory body of the CoO including any documents required by the regulatory body in the country of origin subsequent to the CoO Design submission. The information should demonstrate the acceptability of the referenced Design, and include an independent Safety verification of the same. In addition, the applicant/ Licensee should provide information relating to a comparison between the proposed Design and the referenced Nuclear Facility Design approved by the CoO, including:

- a. a comparison of the applicable Authority requirements and acceptance criteria to the corresponding country of origin requirements and acceptance criteria with an explanation as to why compliance with the country of origin requirements and criteria is sufficient to demonstrate compliance with Authority requirements and criteria;
- b. identification of any Design or operational related changes that have been made to the SSC or related to departures from the reference nuclear power plant (or CoO) from what was reviewed and accepted by the country of origin including an explanation as to their impact on compliance with Authority requirements;
- c. identification of any new Safety issues, standards or requirements related to the SSC or related to departures from the reference nuclear power plant (or CoO), that have been identified since the country of origin approval including an explanation as to their impact on compliance with Authority requirements;
- d. confirmation that the Design conditions (e.g. temperature, humidity, Design life, seismic, operational characteristics) for the SSC or related to departures from the reference nuclear power plant (or CoO), proposed for reliance in the country of origin, envelop the conditions expected in the State;
- e. confirmation that the technology reviewed by the regulatory body of the CoO is the same as that proposed for the State;
- f. confirmation that the risk significance of the SSCs or related to departures from the reference nuclear power plant (or CoO), proposed for reliance in the country of origin, has not substantially changed from the referenced Design; and



- g. an Independent Safety Verification of the above conclusions, as required by Article (6) of FANR-REG-03 (Reference 16) and in IAEA Safety Standards GSR Part 4 on “Safety Assessment for Facilities and Activities” (Reference 17).

Supplement 2. Safety issues and use of OPEX (REG 06, Article (6), Para 8 and REG14, Article (5), Para 8)

The applicant/ Licensee should provide a description of how recent lessons learnt and experience from similar nuclear power plants (especially in the country of origin), scientific and technical developments, applicable worldwide operational experience as well as the results of any relevant research on protection and Safety have been applied to resolve potential Safety issues. The applicant/ Licensee should demonstrate how updating the regulations, codes and standards applied to the referenced Nuclear Facility have been taken into account in the Design, Construction and future Operation of the proposed Nuclear Facility.

Format of the SAR

Article (9)

The applicant/ Licensee should provide to the Authority all presentations of the initial SAR and subsequent revisions in both a paper and electronic format (DVD or equivalent) with a reasonable number of copies to be agreed with the Authority. The electronic SAR should be in PDF or equivalent format and not a scanned version. All elements of the SAR should be clearly legible, including the drawings. The Table of Contents should contain hyperlinks to each item listed. The applicant/ Licensee should store and update the SAR through the use of an effective and secure document control and management system.



References Article (10)

1. FANR Regulation 06 for an Application for a Licence to Construct a Nuclear Facility (FANR-REG-06)
2. FANR Regulation 14 for an Application for a Licence to Operate a Nuclear Facility (FANR-REG-14)
3. IAEA Safety Guide No. GS-G-4.1, *Format and Content of the Safety Analysis Report for Nuclear Power Plants*
4. United States Nuclear Regulatory Commission (NRC) Regulatory Guide RG 1.206, *Combined Licence Applications for Nuclear Power Plants*
5. United States Nuclear Regulatory Commission (NRC) Regulatory Guide RG 1.70, *Standard Format and Content of Safety Analysis Reports for Nuclear Power Plants*
6. United States Nuclear Regulatory Commission (NRC) Regulatory Guide RG 1.68, *Initial Test Programs for Water-Cooled Nuclear Power Plants*
7. FANR Regulation 16 for Operational Safety including Commissioning (FANR-REG-16)
8. FANR Regulation 01 for Management Systems for Nuclear Facilities (FANR-REG-01)
9. FANR Regulatory Guide 002 on Application of Management Systems for Nuclear Facilities (FANR-RG-002)
10. FANR Regulation 08 for the Physical Protection for Nuclear Materials and Nuclear Facilities (FANR-REG-08)
11. FANR Regulation 09 for the Export and Import Control of Nuclear Material, Nuclear Related Items and Nuclear Related Dual-Use Items (FANR-REG-09)
12. FANR Regulation 10 for the System of Accounting for and Control of Nuclear Material and Application of Additional Protocol (FANR-REG-10)
13. FANR Regulatory Guide 014 on Implementation of the System of Accounting for and Control of Nuclear Material at Nuclear Facilities (FANR-RG-014)



14. IAEA Design Information Questionnaire, (DIQ) SG-FM-1098-06
15. FANR Regulation 21 for Decommissioning of Facilities (FANR-REG-21)
16. FANR Regulation 03 for the Design of Nuclear Power Plants (FANR-REG-03)
17. IAEA General Safety Requirements Part 4 No. GSR Part 4, *Safety Assessment for Facilities and Activities*